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THE DESIGN CHARACTERISTICS OF ADVANCED FOWER REACTOR 1400

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I. Introduction

II. Description of Nuclear Systems

III. Description of Secondary Systems

IV. Safety System Design Characteristics

142 - 940

V. Plant Layout

VI. Conclusions



Nuclear Power Plants in Korea

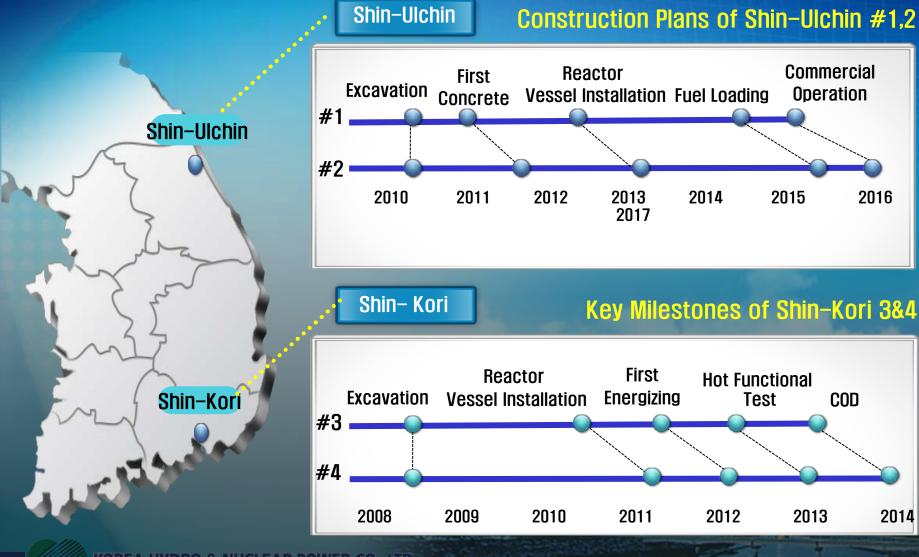
. Introduction

In Operationa 20(17,716) + Under Const 8(9,600) = 23(27,316)



APR1400 Construction Schedules

I. Introduction



Overview of APR1400

. Introduction

Development History of APR1400

Evolutionary ALWR in Korea based on current OPR1000 Design
 Design Certification for the Standard Design

General Requirement

Rated Power : 4000 MWth
Plant Life time : 60 years for major components
Seismic Design : SSE 0.3g
Safety Goal : CDF < 10⁻⁵, CFF < 10⁻⁶

Performance & Economic Goals

Plant Availability : 90%

- Construction Period : 48 Months for Nth Plant
- Economic Goal : 20% advantage over coal



II. Description of Nuclear Systems

Overall Description

- e Rated Power : 4000 MWth
- e 2-Loop PWR :
 - One Reactor Vessel
 - Two Steam Generators
 - Four Reactor Coolant Pumps
 - One Pressurizer
 - Thermal Margin > 10%



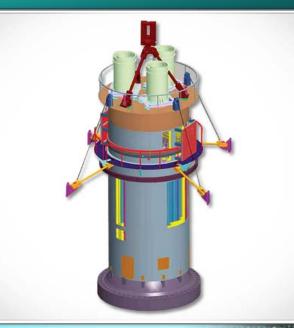
RCS Design Characteristics

II. Description of Nuclear Systems

Hot Leg Temperature Reduction

621°F → 615°F
 To prevent SG tube corrosion

Integrated Head Assembly



Integration of

- Cooling shroud assembly
- CEDM cooling system
- Missile shielding material
- Expected effects of IHA
 - Reducing refueling time
 - Reducing occupational dose
 - Reducing comp. storage area
 - Improving safety for workers



RCS Design Characteristics

Steam Generator

- Increased tube plugging margin : $8 \rightarrow 10\%$
- Corrosion resistant tube material : $1600 \rightarrow 1690$
- Increased secondary inventory to prolong SG dryout time
- Automatic level control for all power level

Pressurizer

Larger steam space to accommodate RCS transients
 Adoption of POSRV instead of PSV + SDS

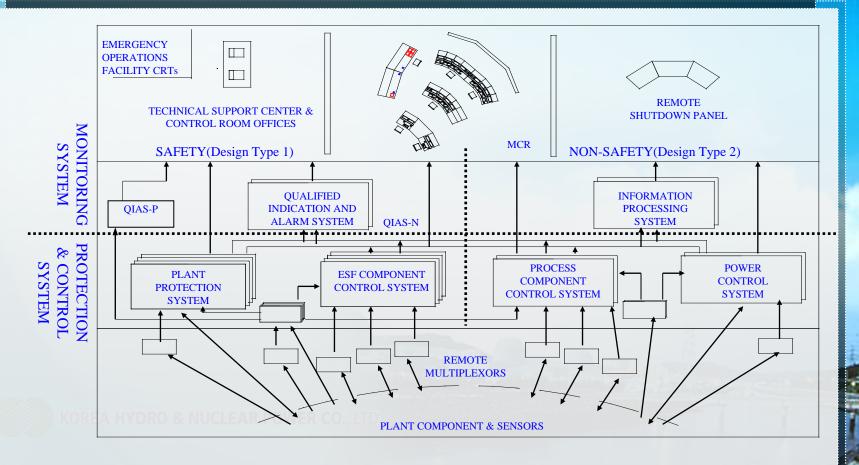
- 4 PSVs + 2 SDS \rightarrow 4 POSRVs
- Over pressure protection + Safety depressurization function
- High reliability



III. Description of Secondary Systems

Design Characteristics of I&C Systems

- Digital Technology & Data communication network
- Open & Standard Architecture
- Defense on Common mode failure
- Operability & Maintenance : Auto test, Self-diagnosis



III. Description of Secondary Systems

Characteristics of MCR Design

- Multiple Compact Workstation MCR
- Large Display Panel
- Soft-Controller : Safety & Non-safety control
- Computerized Procedure Systems
- Adoption of Human Factor Engineering

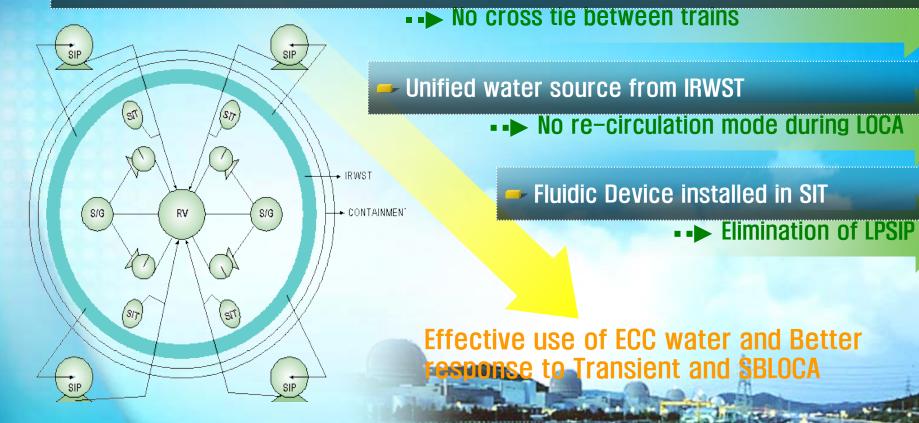


Safety Injection System

IV. Safety System Design Characteristics

Design Characteristics

4 independent trains (Each train consists of 1 SIP and 1 SIT)





Verification of Fluidic Device

IV. Safety System Design Characteristics

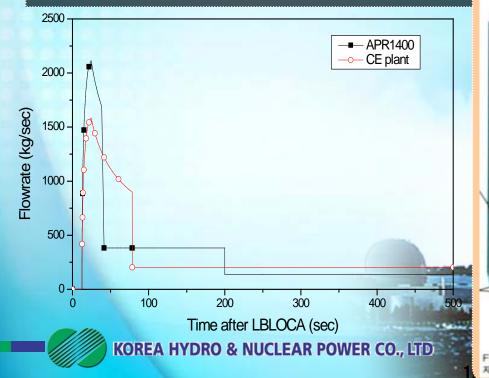
Design Characteristics

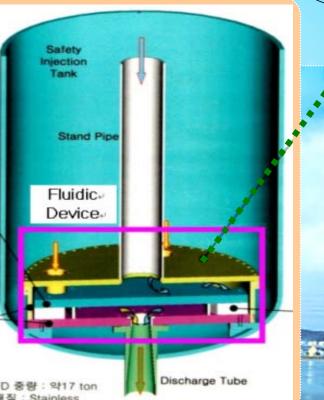
Principles : Vortex flow resistance

- Stand pipe : Low resistance
- Control port : high resistance

Purpose : Extending SIT injection period during LOCA

Effect : Removal of LPSIPs





Verification of Fluidic Device

IV. Safety System Design Characteristics

Test Facility

Actual physical dimension

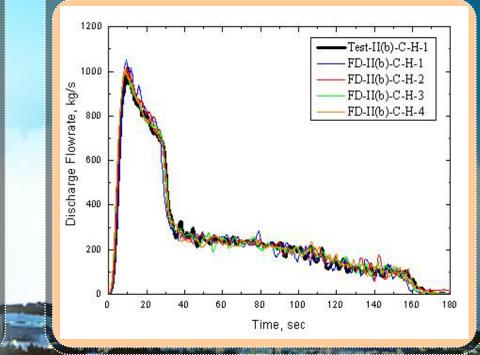
 Inside diameter : 2.74 m
 Total Height : 12.0 m

Full Pressure : 50 bar

Test Results

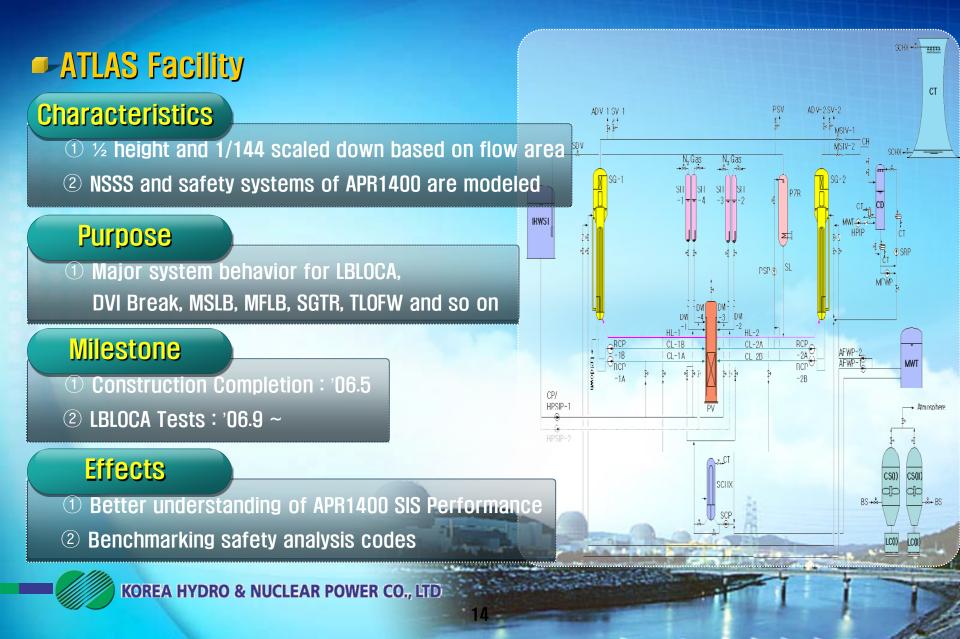
Test Results

- Total 28 tests has been performed
- Performance is fully verified
- Detailed design spec. is finalized



Integral Test Loop

IV. Safety System Design Characteristics



IV. Safety System Design Characteristics

Functions

IRWS

- Supply cooling water during refueling
- Supply the water source to safety injection and containment spray systems during DBA
- Removed recirculation mode because bleeded coolant is collected in IRWST through HVT (Hold-up Volume Tank)
- Supply heat sink during rapid depressurization of RCS and feed and bleed operation
- Supply water source for reactor cavity flooding system

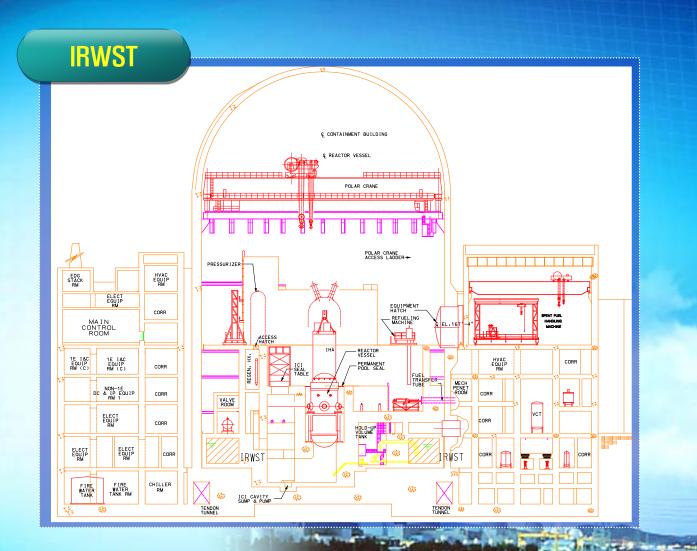
Design characteristics

- Collect the coolant from POSRV to IRWST through sparger
- Removing of Pressurizer Relief Tank (PRT)
- Reduce operator's burden with simplified operation mode
- Reduce containment penetrations



IV. Safety System Design Characteristics

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External Cooling of RPV (ERVC)

ERVC

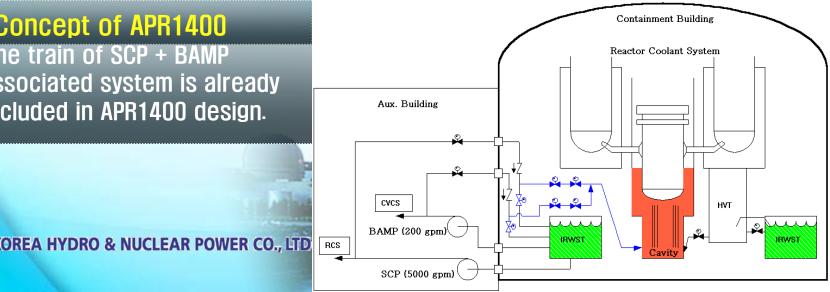
- Strategy to retain corium in vessel by submerging reactor vessel exterior with water
- Used in Lovisa and AP600 and AP1000
- Theofanous conducted a structured study on its performance
- Chosen as a key accident management strategy for APR1400

Accident Management Strategy of APR1400

- If water can be injected to reactor cavity : ERVC
- If not (eg. SBO): corium spread to reactor cavity and cooling from IRWST using gravity head.

ERVC Concept of APR1400

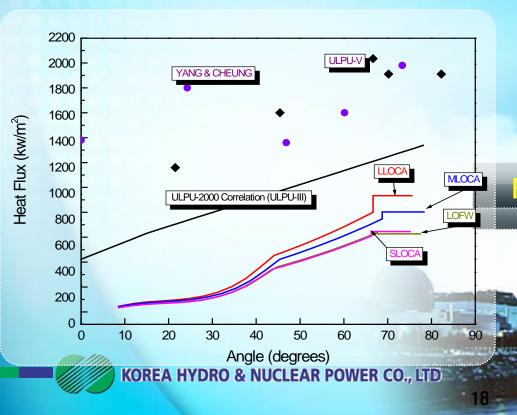
- One train of SCP + BAMP
- Associated system is already included in APR1400 design.



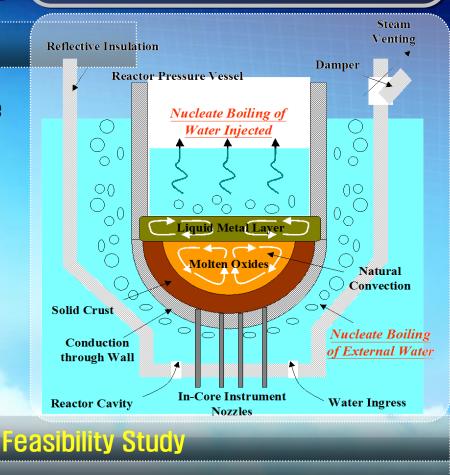
Performance of ERVC in APR1400

RPV Insulation

- Passage way for the water to cool the hot reactor vessel
- Natural circulation path for the twophase flow and influence CHF
- Streamlined insulation design for APR1400 is in progress



IV. Safety System Design Characteristics

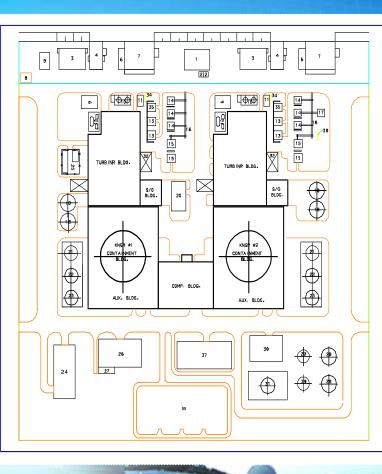


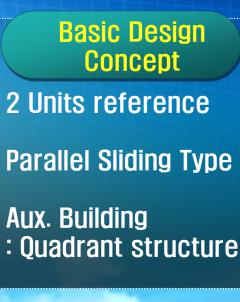
- 4 Major scenarios using MAAP
- Margin for APR1400 shows the usefulness of the ERVC strategy

V. Plant Layout

Power Block : NI & TI

- NI : Reactor Building, Aux. Building, Compound Building
- TI : Turbine Building, SwitchGear



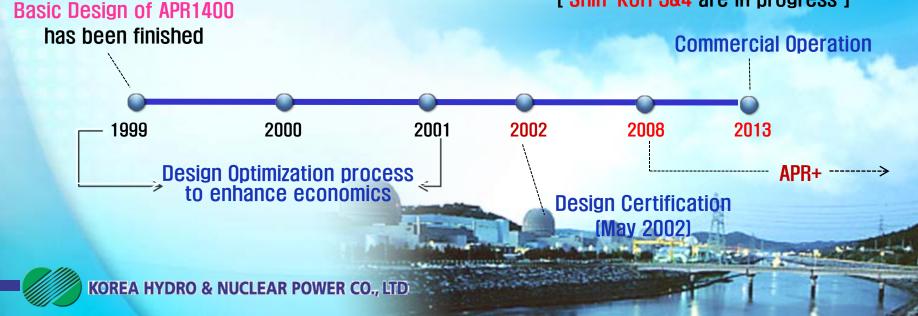




VI. Conclusions



First commercial NPPs of APR1400 [Shin-Kori 3&4 are in progress]



Thank you I

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